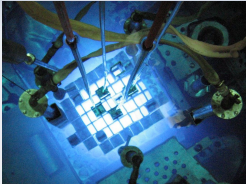


Kinetic Parameters Estimation in a MTR Research and Production Reactor in Subcritical States

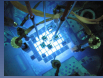


P.A. Bellino A. Gomez G. Estryk C. Grant

National Atomic Energy Commission, Department of Nuclear Energy, Argentina

International Conference on Research Reactors:
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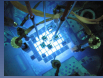
14-18 November 2011, Rabat, Morocco



Objectives

Kinetic parameters estimation

- Inverse kinetics as a reactimeter in subcritical states (monitoring a refueling operation).
- Determination of the strength of the neutron source (point kinetic model).
- Characterization of the neutron source. Photoneutron effectiveness estimation (γ^{ph}).
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- Reactivity and power estimations (30 *mW* to 200 *mW*).



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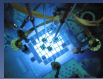
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Photoneutrons

- Produced in the reaction $D(\gamma, n)H$ for $E_\gamma > 2,23 \text{ MeV}$
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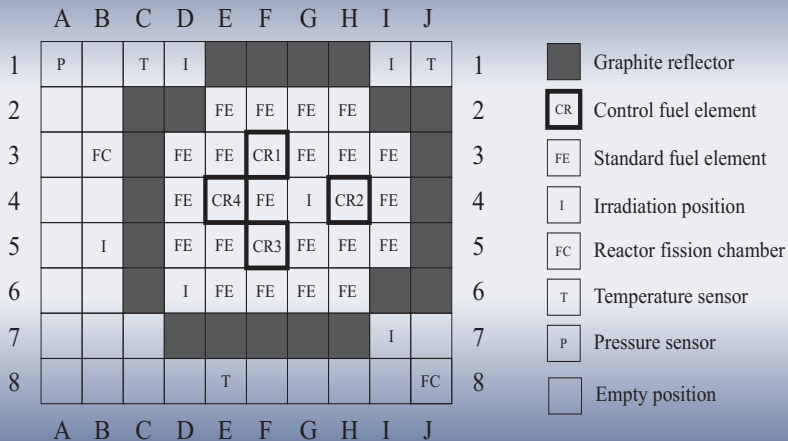
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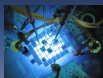
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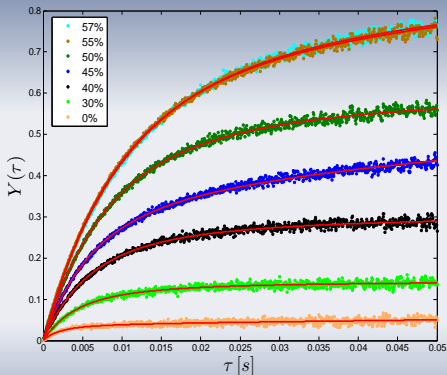


RA-3 core configuration





Neutron noise technique



α -Feynman method

$$Y(\tau) = \frac{\epsilon D}{\alpha^2 \Lambda^2} \left(1 - \frac{1 - e^{-\alpha \tau}}{\alpha \tau} \right)$$

D : Diven factor

Λ : Neutron generation time

ϵ : Absolute efficiency (\sim Power)

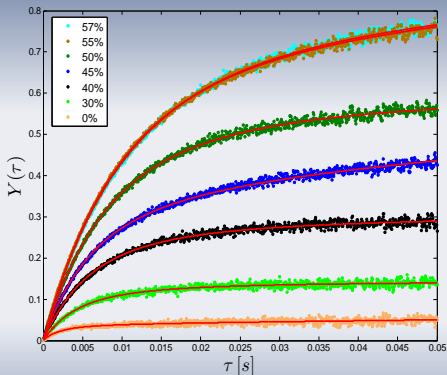
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Count rate: $1 \cdot 10^5 \text{ cps} < R < 7 \cdot 10^5 \text{ cps}$

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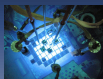
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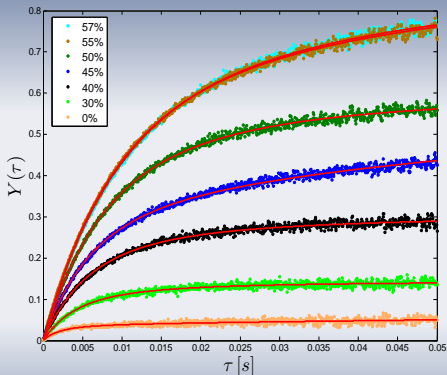
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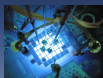
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Estimation of α_c

At each subcritical state:

$$\alpha = \alpha_c + \frac{\tilde{\Sigma}}{R}$$

By measuring α and R at different subcritical states a linear fit can be performed to obtain α_c .

$$\alpha_c = (106 \pm 1)s^{-1}$$



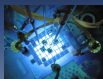
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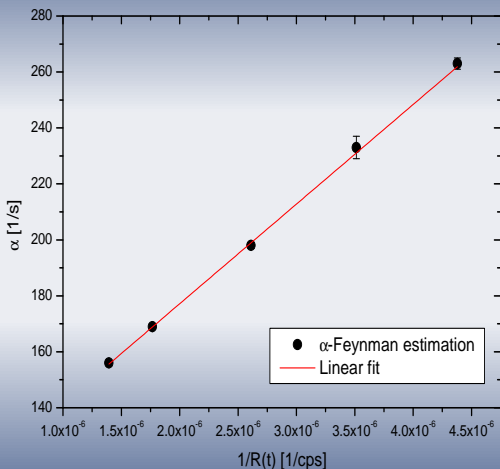
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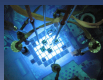
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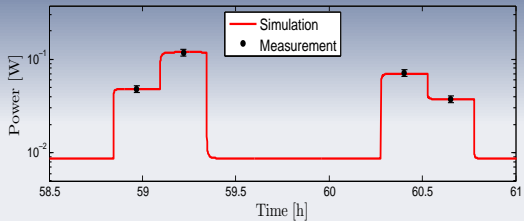
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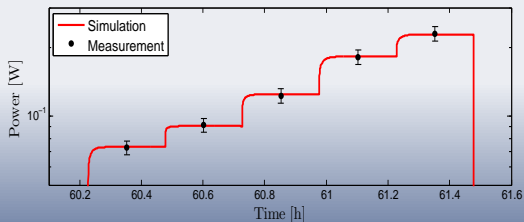


Results

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Parametric variation of γ^{ph} in the point kinetics equations until the best fit of the simulation to the measured power is found.



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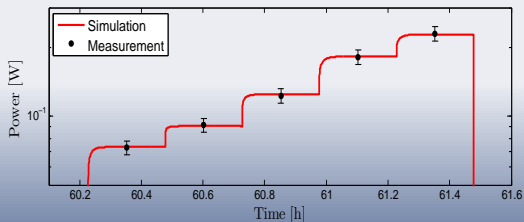
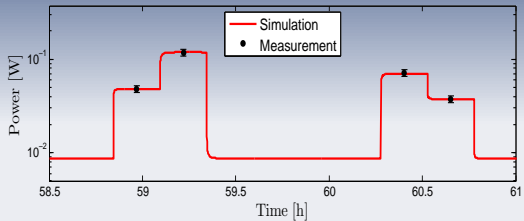
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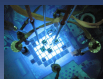


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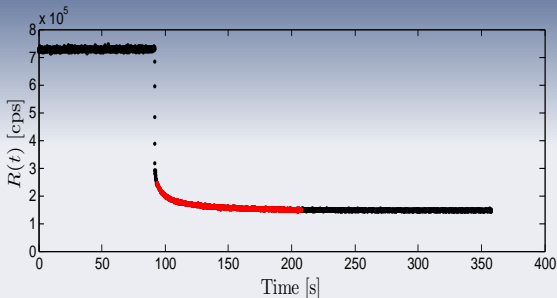
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Least square inverse kinetics method



Rod-drop between two subcritical states.

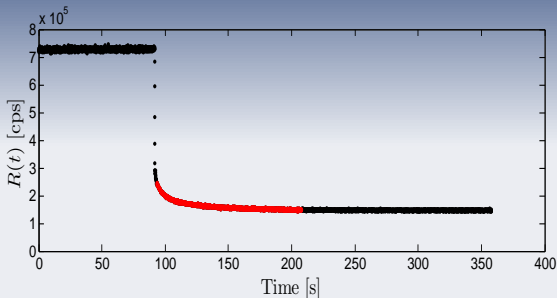
Transient evolution is fitted with a linearized model (LSIKM).

$$R(t) = \frac{\Lambda^*}{\beta_f - 1} \tilde{Q}(t) - \frac{\Lambda^* \tilde{S}}{\beta_f - 1}$$

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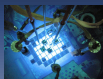


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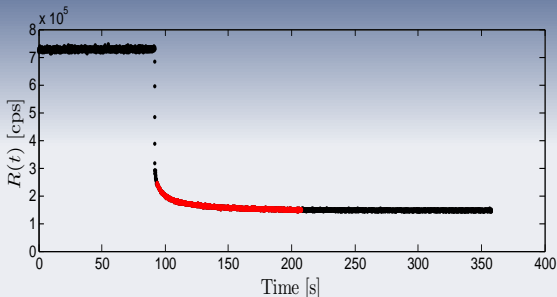
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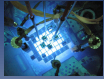


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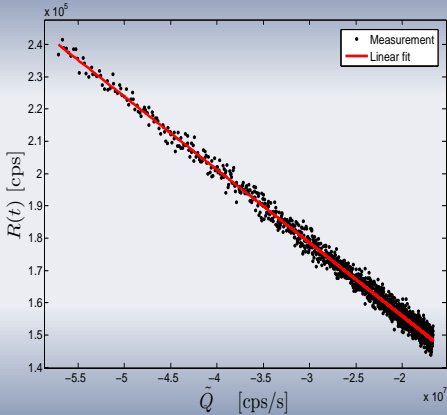
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Estimated values

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$$\tilde{S} = (6,90 \pm 0,04) 10^6 \text{ cps/s}$$

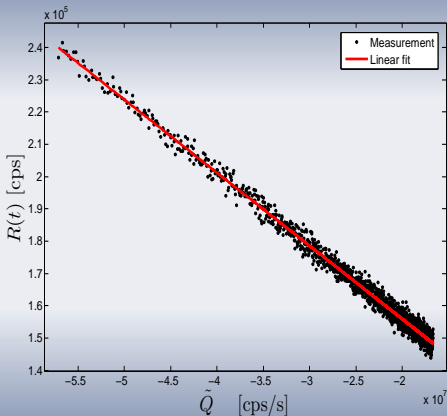
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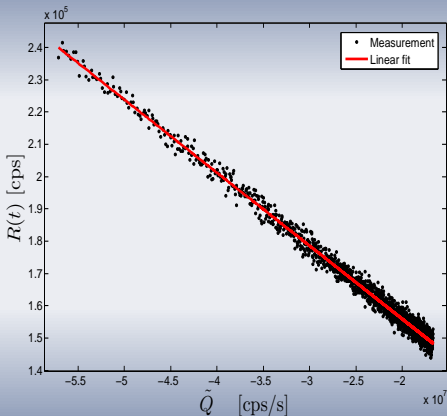
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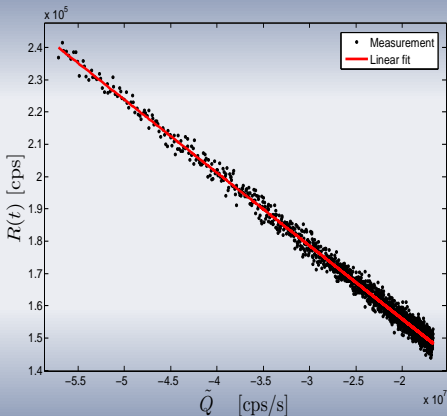
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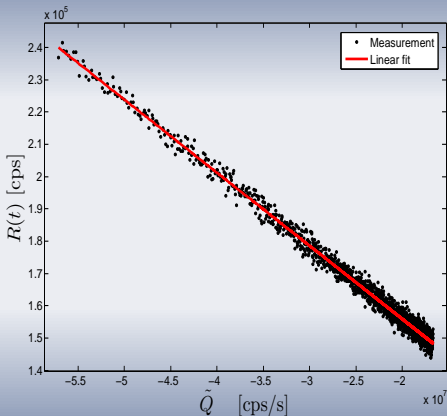
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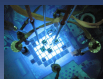
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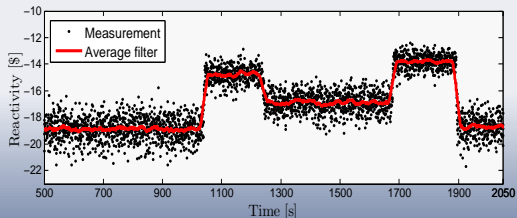
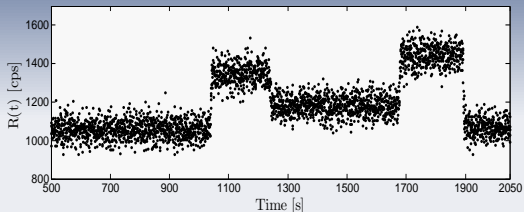
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With the estimation of \tilde{S} , the reactivity can be obtained measuring the count rate during a refueling operation.

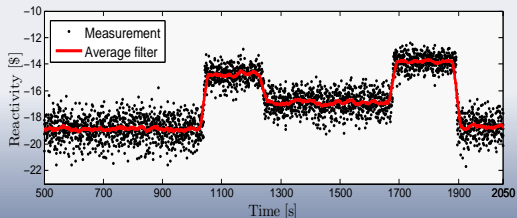
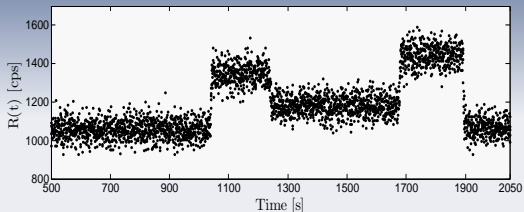
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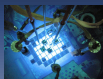
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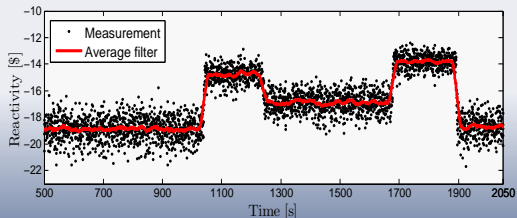
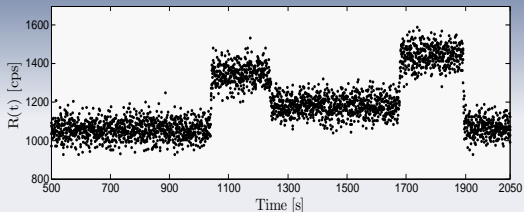
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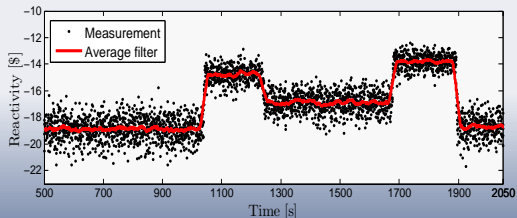
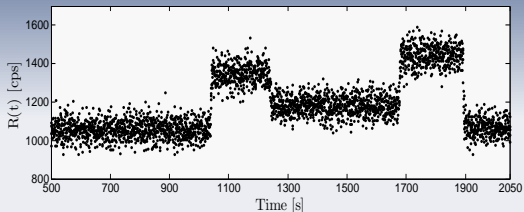
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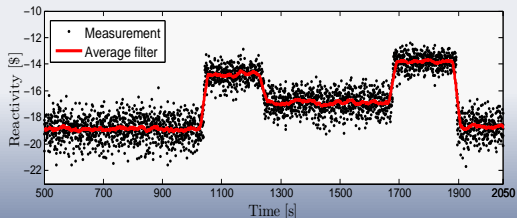
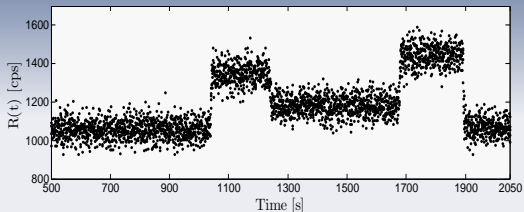
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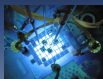
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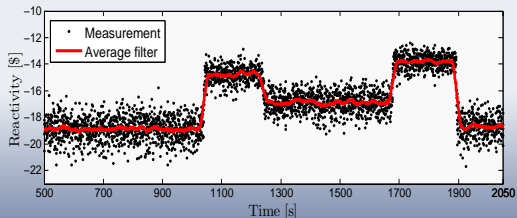
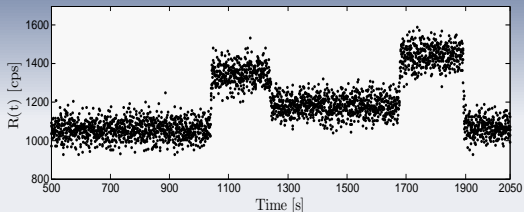
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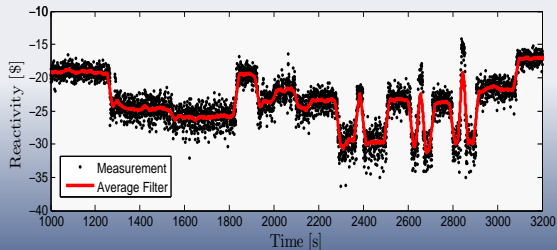
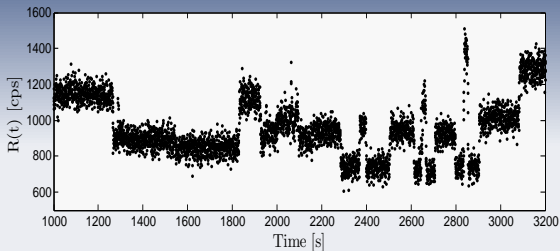
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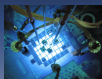
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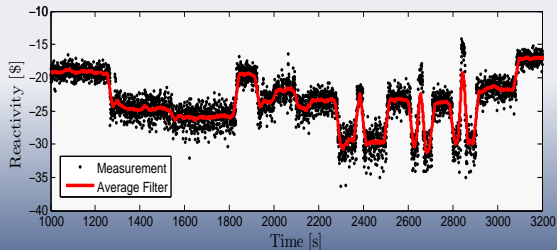
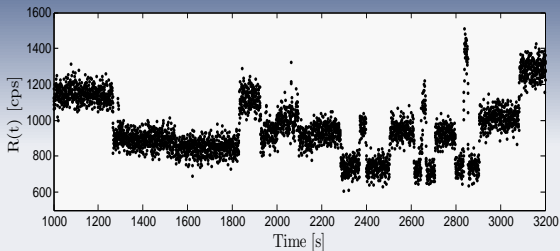
Core 212 to 213

Extraction of burned
FE, rotation and
entering fresh FE

At these highly
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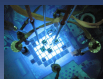
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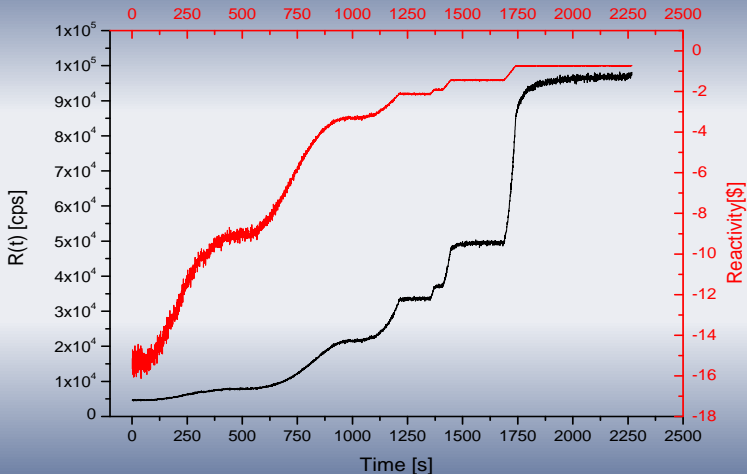
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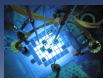
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Approach to critical





Numerical model

Diffusive code PUMA

Using a homogeneous neutron source in all the fuel channels
(First step)

- As the magnitude of the neutron source was not known in advance, all the comparisons were made relative to the first state.
- A refueling operation was calculated.



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Using a homogeneous neutron source in all the fuel channels
(First step)

- As the magnitude of the neutron source was not known in advance, all the comparisons were made relative to the first state.
- A refueling operation was calculated.



Numerical model

Diffusive code PUMA

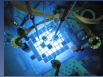
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Results from core 212 to 213

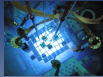
Core configuration i	Experimental		PUMA		
	R_i/R_0	ρ_0/ρ_i	ϕ_i/ϕ_0	P_i/P_0	ρ_0/ρ_i
Core 212 0	1	1	1	1	1
1	0.78 (3)	0.8 (1)	0.784	0.910	0.928
2	0.74 (3)	0.7 (1)	0.740	0.788	0.809
3	0.98 (4)	1.00 (5)	0.962	0.878	0.885
4	0.82 (3)	0.8 (1)	0.806	0.915	0.939
5	0.81 (3)	0.8 (1)	0.787	0.884	0.901
6	0.88 (4)	0.9 (1)	0.821	0.783	0.787
Core 213 7	1.11 (4)	1.12 (5)	1.113	1.090	1.081



Numerical model

Future improvements

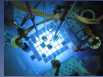
- Calculate the photoneutron source.
- ORIGEN with the information of each FE.
- MCNPX to obtain the photoneutron source.
- MCNPX/PUMA for the neutronic problem.



Numerical model

Future improvements

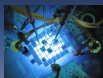
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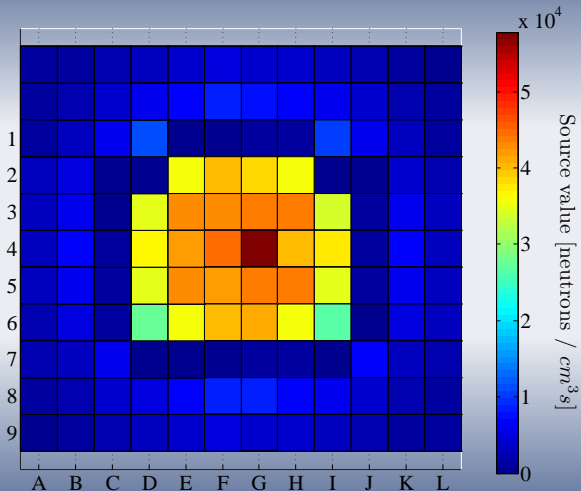
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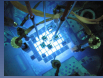
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Photoneutron source (MCNPX)





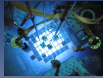
Conclusions

- Measurements at subcritical states with high gamma background and ^{135}Xe .
- With the neutron noise technique reactivities were estimated between -6 and $-0,5$ dollars. Power between 30 mW and 200 mW .
- First estimations of the photoneutron effectiveness $\gamma^{ph} = (1,12 \pm 0,06) 10^{-4}$
- Estimation of the source strength value (\tilde{S}) that appears in the point kinetics equation.
- Using the inverse kinetics as a subcritical reactivity meter
- Monitoring a refueling operation and an approach to critical with a subcritical reactivity meter.
- Future measurements with two or more detectors. Detailed studies of spatial effects (correction factors).



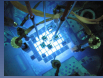
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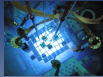
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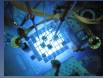
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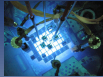
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Thank you for your attention

